



DE LA RECHERCHE À L'INDUSTRIE

Complementary study on Superconducting Toroidal Field Coils of JT-60SA

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Europe & Japan collaboration for developing a thermo-nuclear fusion reactor upgraded with 18 superconducting Toroidal Field Coils (TFC)

Supercritical Helium
($T_c = 5.20K, P_c = 2,27 \text{ bar}$)

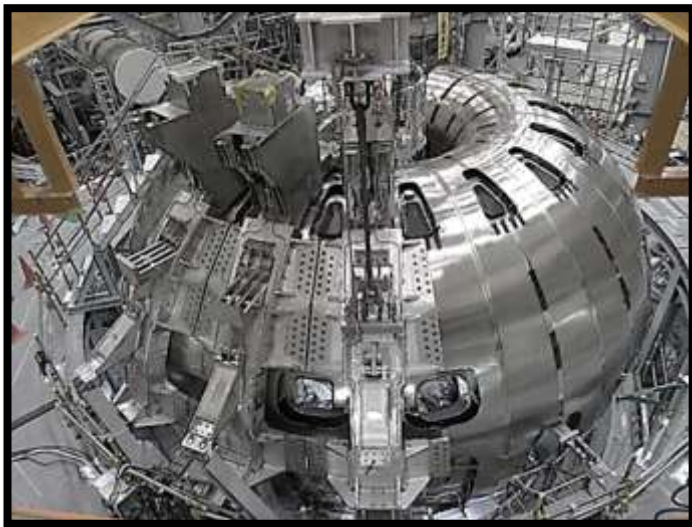


Fig.1. View of the JT-60SA tokamak machine

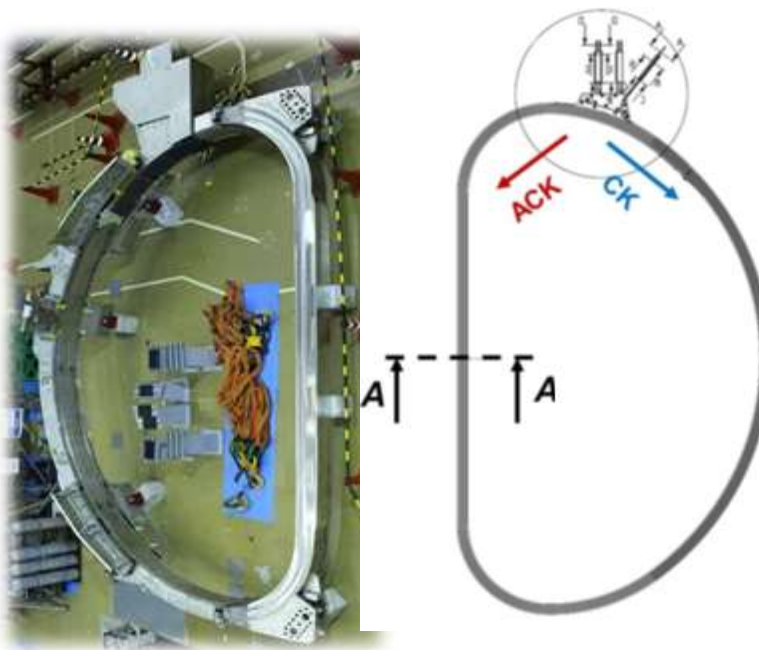


Fig.2. Upper view of one TFC

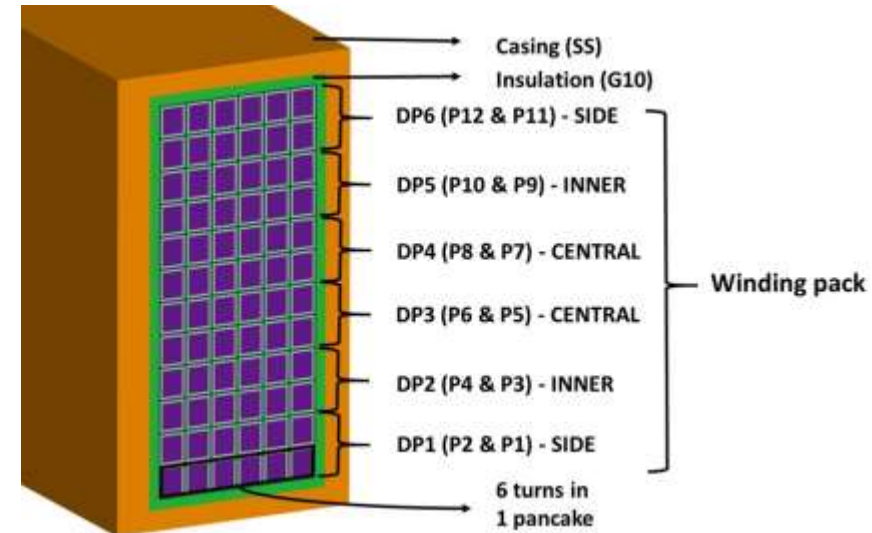
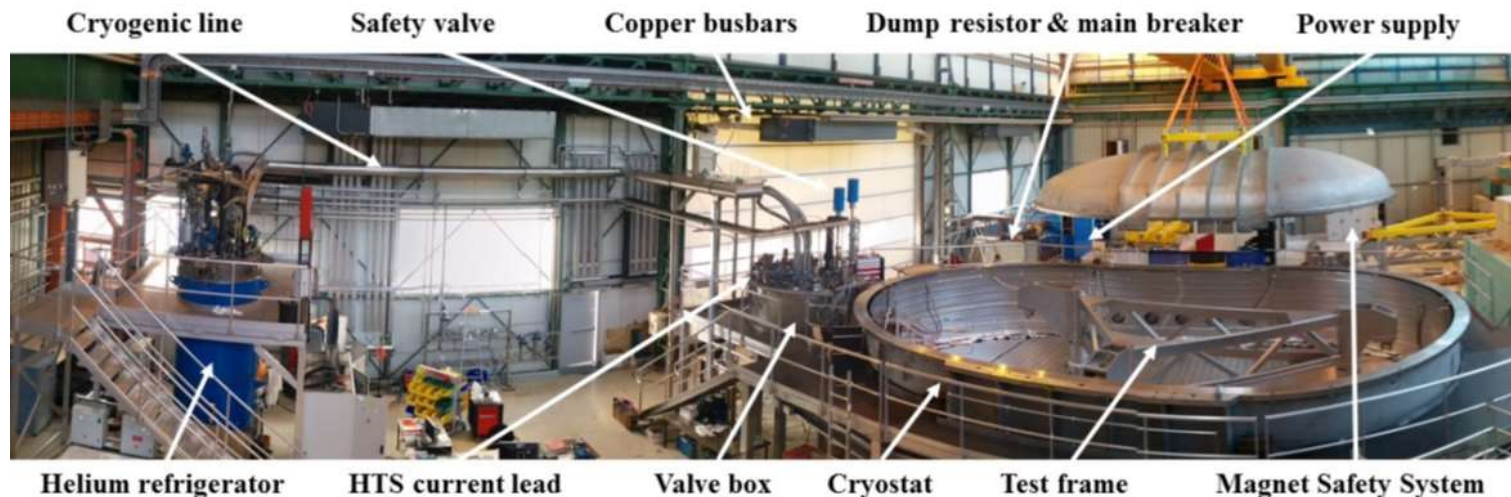


Fig.3. A-A view

(Courtesy of Huang Y. et al, "Quench tests analysis of the first JT-60SA Toroidal Field coils", 2017 Fusion Eng. Des. 124 147-152)



CEA Saclay

Fig.4. Panorama photo of the Cold Test Facility and cryostat

Previous Study: experimental tests (+ numerical modelling)

Number of tested coils: All 18 TF coils, in a single coil configuration

Test configuration: Nominal (electrical and cryogenic) operating

conditions: ($I_{nom} = 25.7 \text{ kA}$, $T_{He} = 5 \text{ K}$)

+

1 Quench protocole

Main focus: Quench dynamics in CICC cooled with supercritical Helium flow ($T_c = 5.20 \text{ K}$, $P_c = 2,27 \text{ bar}$)

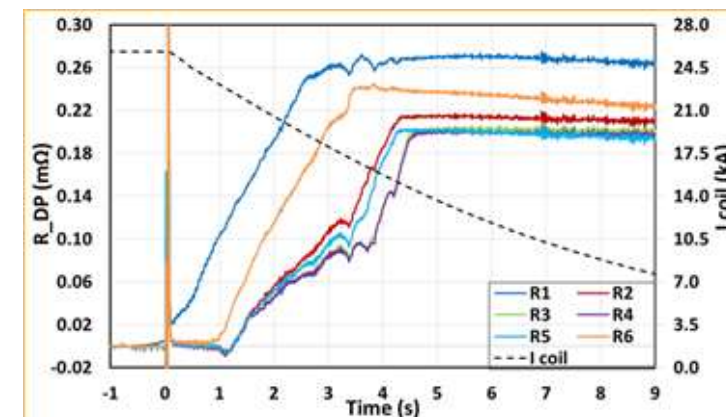


Fig.5. Example of a quench evolution for one of the tested TFC of JT-60SA

► **1. CONTEXT**

- a. JT-60SA magnet
- b. Cold Test Facility Activity
- c. Additional Tests Activity at CTF

► **2. DESCRIPTION OF QUENCH EXPERIMENTS**

- a. Experimental setup & Instrumentation
- b. Experimental quench protocol
- c. Experimental quench resistance computation: pick-up coil method
- d. Experimental study cases for numerical analysis: Observed dynamics

► **3. NUMERICAL MODELLING & ANALYSIS OF QUENCH EXPERIMENTS**

- a. Simplified 1D model
- b. Interturn
- c. Numerical analysis of the experimental quench dynamics

► **4. CONCLUSION & OUTLOOK**

1.c. ATA: Additional Tests Activity at CTF

Additional R&D program



F4E

Thermohydraulic tests (TFC19)

TFC spares: TFC02 & TFC19



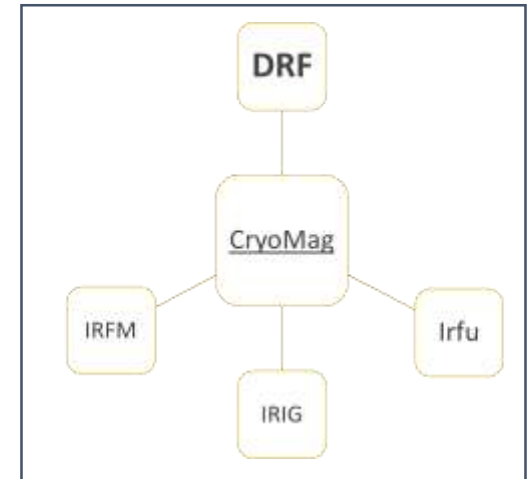
F4E

Quench tests (TFC02)



CryoMag

Development of numerical models (ongoing)



Activity Name	Coil	Relevant Parameter
ATA01	TFC19	Casing Temperature Stabilisation
ATA02	TFC19	Pulsed Heat Input
ATA03	TFC19	Higher mass flow rate hydraulic test
ATA04	TFC19	Casing /Winding Pack Thermal Coupling
ATA05	TFC02	Quench under different inlet temperature dynamics
ATA06	TFC02	Lower mass flow quench tests
ATA07	TFC02	Higher mass flow quench tests
ATA08	TFC02	Sensitivity of detection time
ATA09	TFC02	Sensitivity of detection thresholds (voltage)
ATA10	TFC02	Lower Current Quench tests
ATA11	TFC02	Local Heat Input Quench

ATAs
Identification

ATAs
purpose

➤ For F4E: study of TF coils behaviour in off-nominal working conditions of the Tokamak.

➤ For CEA/CryoMag:
To better understand CICC's behavior
To isolate specific parameters influence
To closely observe coupled phenomena
To benchmark numerical models

Experimental tests considered in numerical study cases

► **1. CONTEXT**

- a. JT-60SA magnet
- b. Operational tests
- c. Additional Tests Activity at CTF

► **2. DESCRIPTION OF QUENCH EXPERIMENTS**

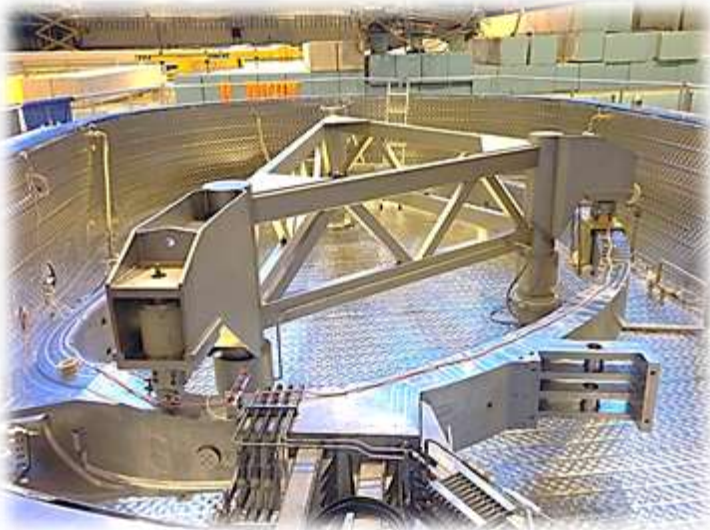
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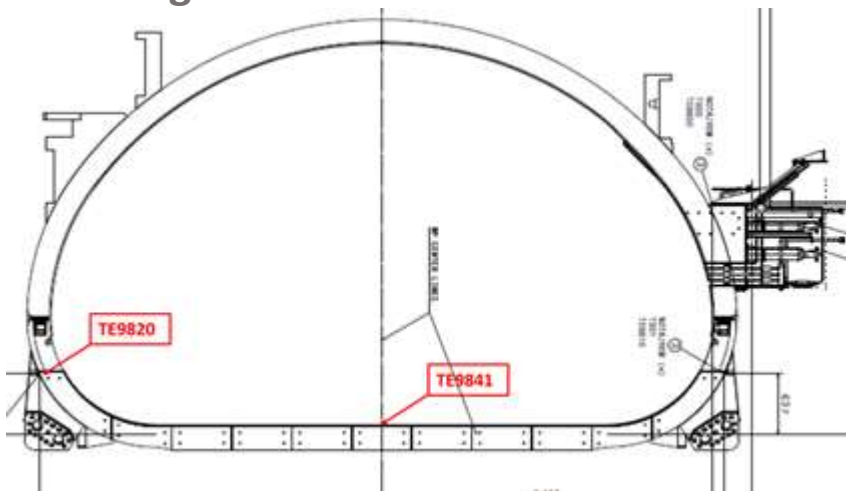
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► **4. CONCLUSION & OUTLOOK**

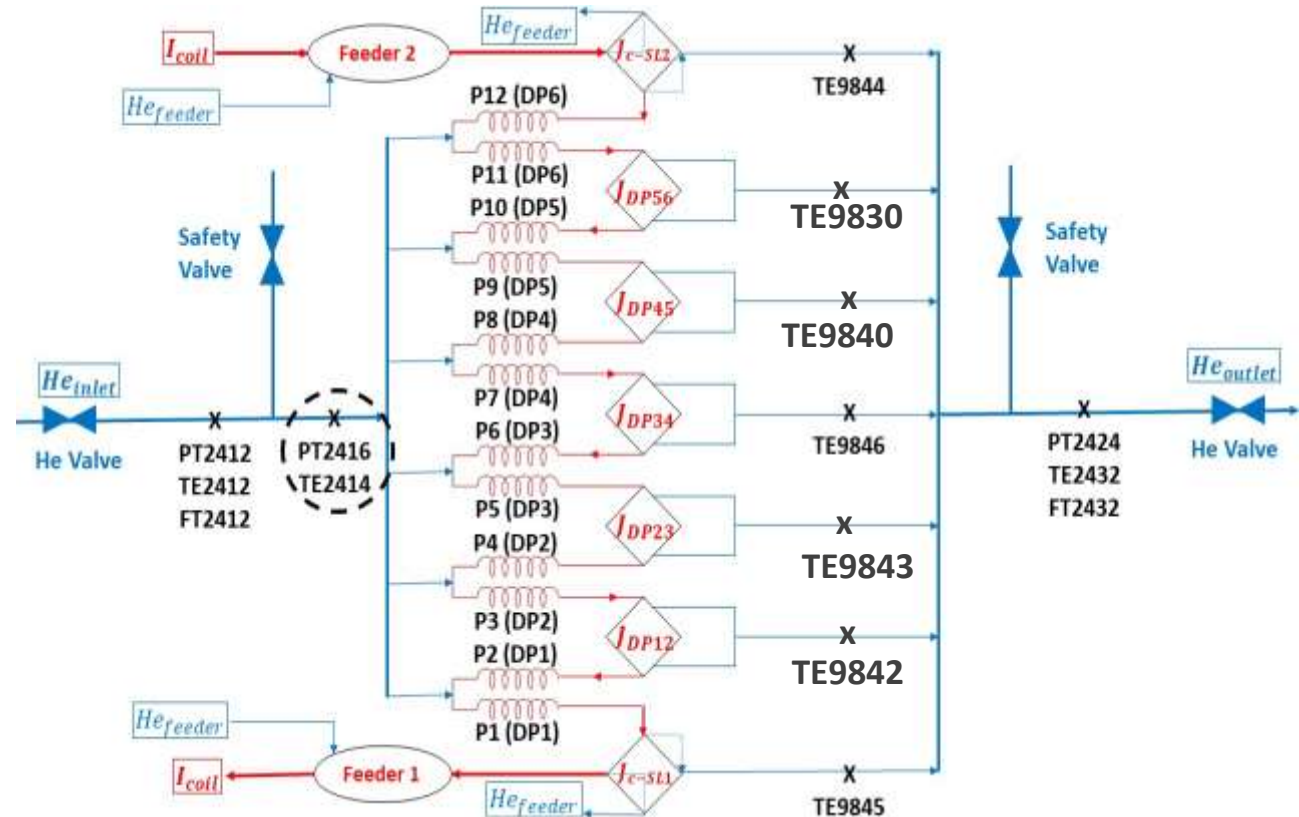
TFC inside the cryostat at CTF



Casing instrumentation of TFC



TF coil Instrumentation at CTF for quench tests



- ▶ PT: pressure transmitter
- ▶ TE: temperature gauges
- ▶ FT: mass flow rate transmitter
- ▶ J_{DP} : interpancake joint
- ▶ J_{C-SL} : connecting joint (coil-superconducting feeder)

2.b. Experimental quench protocole in three steps

1

Coil energizing phase
(steady state)

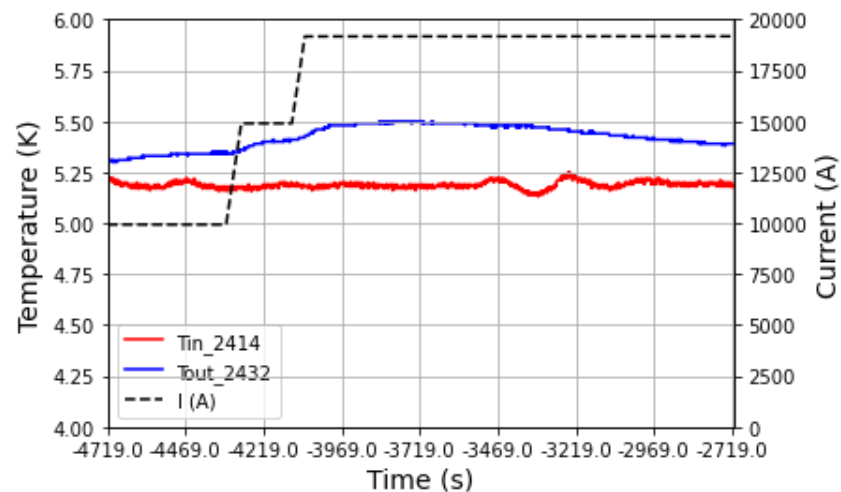


Figure 1-Temperature and current evolution during the Helium steady-state

Quench detection
of 100mV during
100ms



3

Current Fast discharge
(Fast transient state)

Protocole example for lower current quench test ($I=19.5$ kA)

2

Temperature-increasing phase
(slow transient state)

Second slower slope

First rapid slope

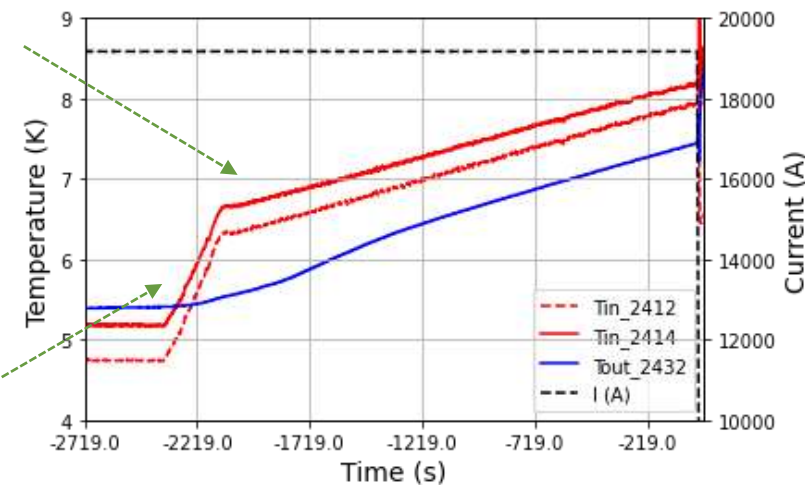
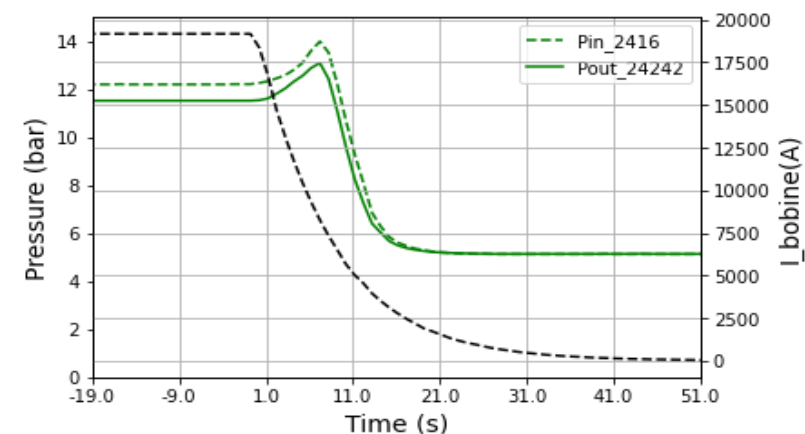


Figure 1-Temperature and current evolution during Helium slow transient state



1. Electrical data acquisition:

The experimental DP's **quench development resistance** **obtained** from the **voltage measurements**.

Time evolution of the voltage U_{DP} is due to a competition between the inductive voltage $U_{DP\ ind}$ and the quench resistive voltage $U_{DP\ quench}$:

$$\begin{aligned} U_{DP} &= U_{DP\ ind} + U_{DP\ quench} \\ &= L \frac{dI}{dt} + IR_{DP} \end{aligned}$$

How to deduce the quench resistive contribution from U_{DP} ?

2. Pure fast discharge (PFD):

no quench in this coil

The voltage corresponds to a purely inductive voltage only depending on the current variation.

3. Computation with the pick-up coil inductive voltage

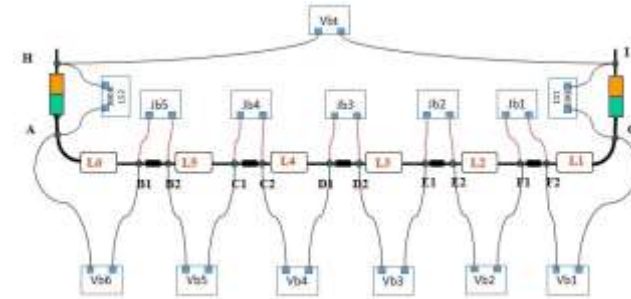
« Method » *Computation with the pick-up coil inductive voltage* » (**Huwai Y. Method**)

- During the FD, the rapid variation of the coil current produces the magnetic field variation through the pick-up coil and induces a pure inductive voltage in it.
- DP inductive voltage is proportional to the one of the pick-up coil according to the relation to the self-inductances.

L = global inductance of the DP including the self-inductance and the mutual one

$$\gamma_i = \left(\frac{U_{DP\ i\ mes}}{U_{pickup}} \right)_{PDF} = \left(\frac{L_i \frac{dI}{dt}}{U_{pickup}} \right)_{quench}$$

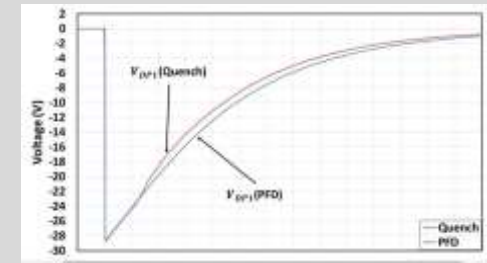
$$R_{DP\ i} = \left(\frac{U_{DP\ i\ mes} - \gamma_i U_{pickup}}{I} \right)_{quench}$$



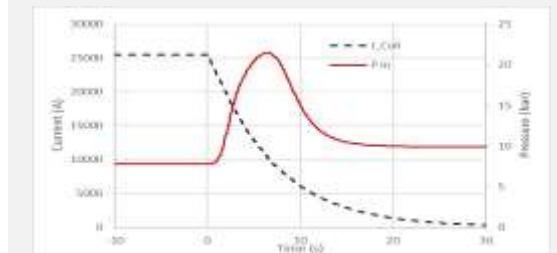
Pick-up coil



Pure Fast Discharge



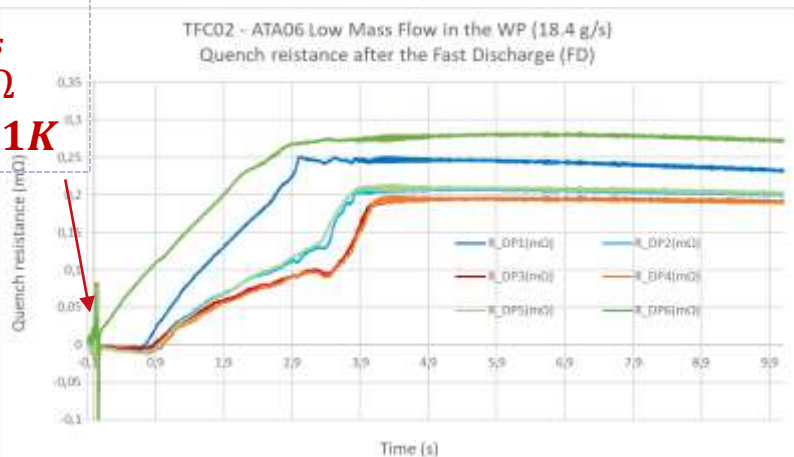
Current fast discharge during the quench



Lower Mass Flow in the WP: 18.4 g/s

$$R_{FD-0,1s} = 16 \mu\Omega$$

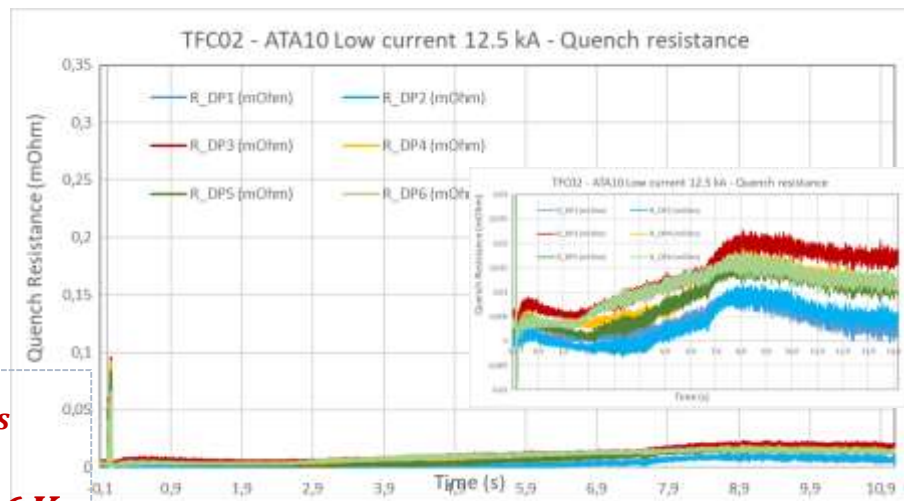
$$T_{FD} = 7,61K$$



Time delay prior
to the current
dump = **100 ms.**
Voltage = **100 mV**

Lower Current :

12.3 kA



$$R_{FD-0,1s} = 5 \mu\Omega$$

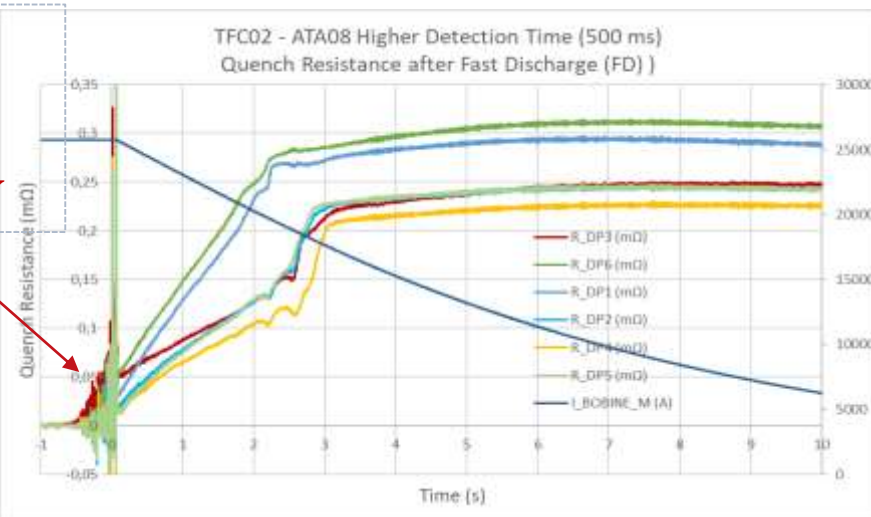
$$T_{FD} = 8.36K$$

19.5 kA

Higher Detection Time: $\tau = 500 \text{ ms}$

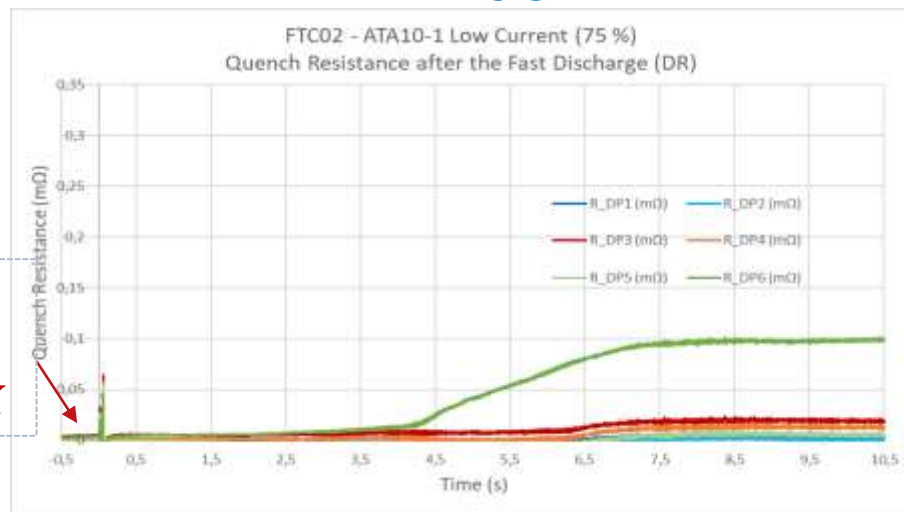
$$R_{FD-0,1s} = 59 \mu\Omega$$

$$T_{FD} = 7.51K$$



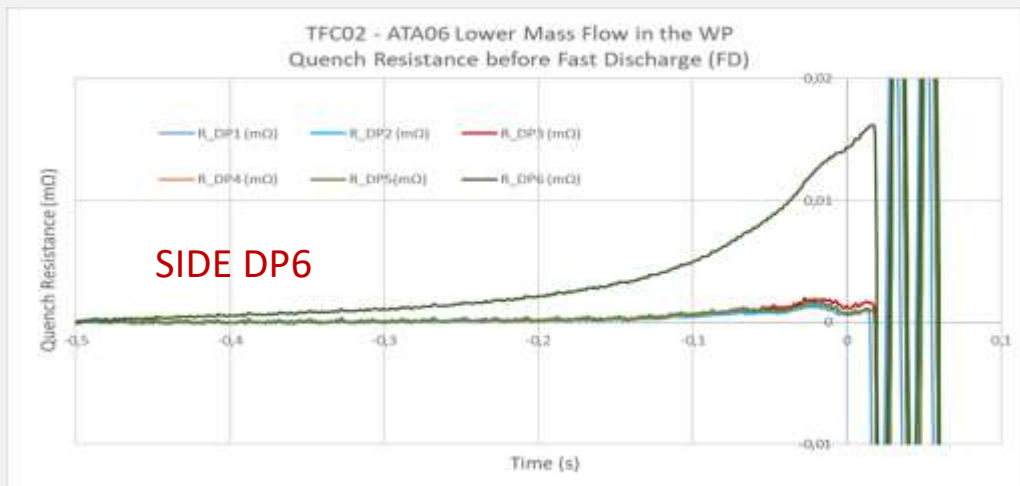
$$R_{FD-0,1s} = 5 \mu\Omega$$

$$T_{FD} = 7.93K$$

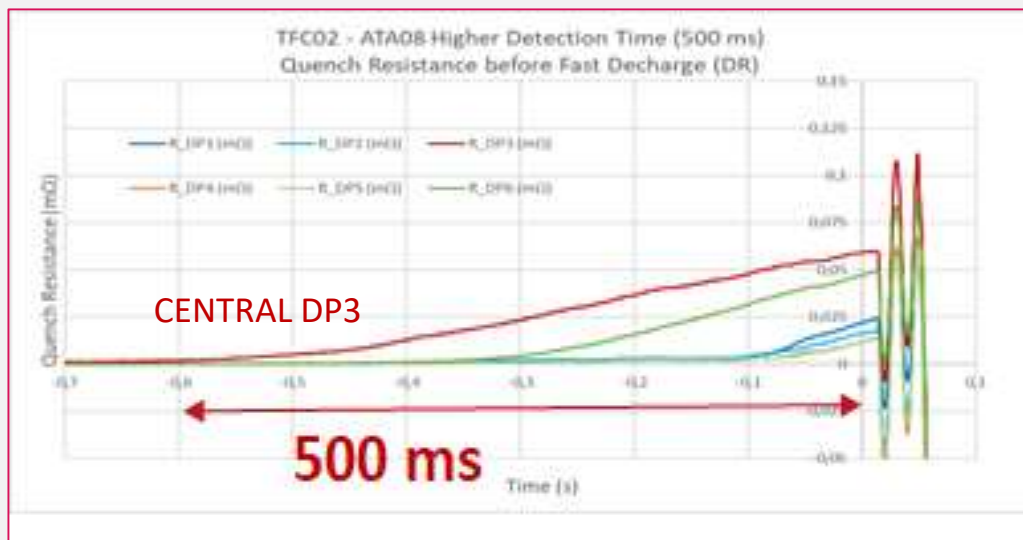


2.c. Experimental study cases for numerical analysis: DPs quench resistance evolution in DR [-0.5; +0.1]s

Lower Mass Flow in the WP: 18.4 g/s

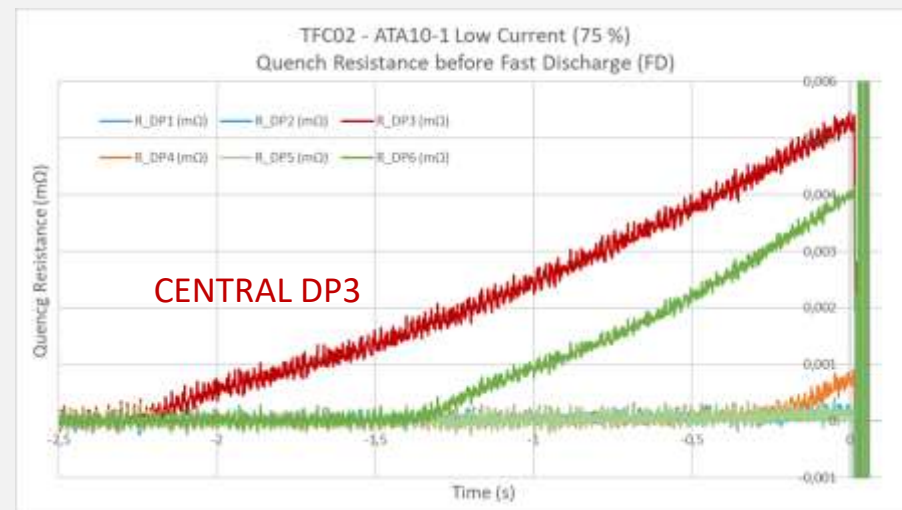


Higher Detection Time: $\tau = 500$ ms



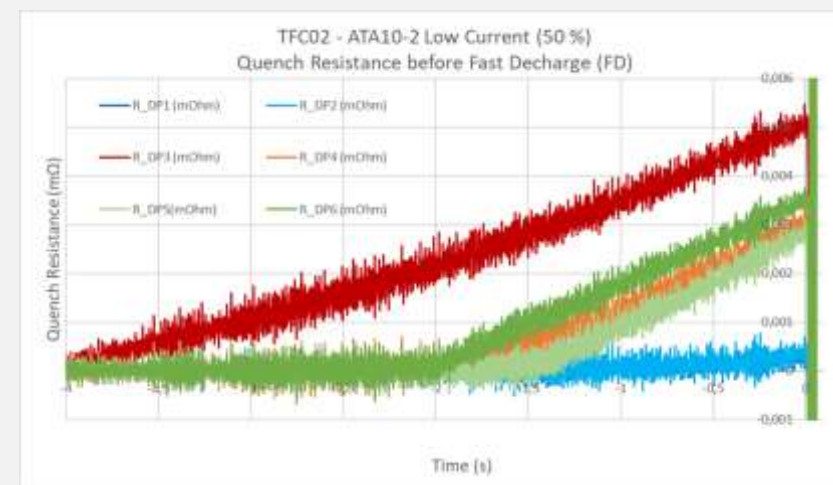
Lower Current :

12.5 kA



Lower Current:

19.5 kA



► **1. CONTEXT**

- a. JT-60SA magnet
- b. Operational tests
- c. Additional Tests Activity: premise & purpose

► **2. DESCRIPTION OF QUENCH EXPERIMENTS**

- a. Instrumentation
- b. Experimental quench protocol
- c. Summary of the quench experimental tests
- d. Experimental study cases for numerical analysis: Observed dynamics

► **3. NUMERICAL MODELLING & ANALYSIS OF QUENCH EXPERIMENTS**

- a. Simplified 1D model
- b. Interturn
- c. Numerical analysis of the experimental quench dynamics

► **4. CONCLUSION & OUTLOOK**

3.a. Thermohydraulic code THEA: simplified model

THEA (Cryosoft) is FE model for the Thermal, Hydraulic and Electrical Analysis of superconducting cables.

► THEA computes the evolution of the temperature, coolant flow and current distribution in one cable in:

- steady state scenarios
- transient (QUENCH) modes.

► **Simplified 1D steady-state model:**

- Thermal components: strands, jacket & isolation

Matériau	Section (mm ²)	Masse volumique (kg/m ³)	Longueur(mm)
NbTi	56.8	6550	113.27
Cu (RRR=130)	180	8960	
Acier inoxydable	176	7900	
G10	100	1948	

- Hydraulic components: supercritical Helium in forced flow

► **Main thermal-hydraulic parameters:**

- $T_{in} = 5.14 \text{ K}$
- $\Delta P = 550 \text{ mbar}$

► **Heat transfer coefficients**

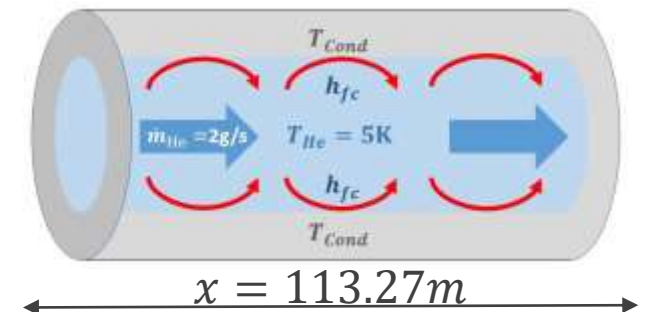
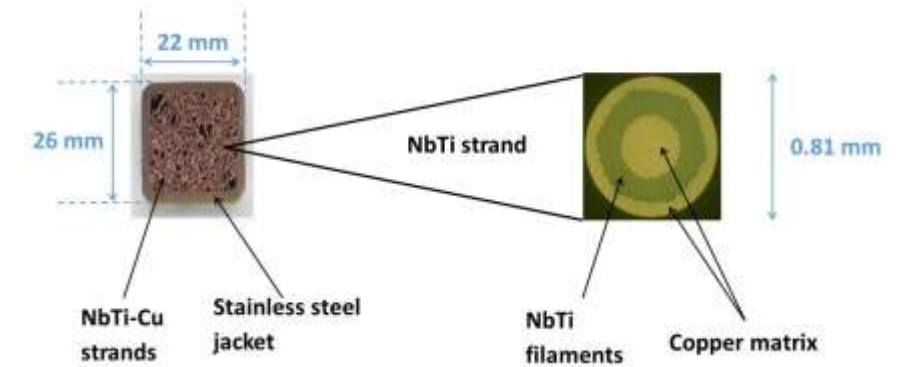
- Dittus-Boelter correlation: $Nu_{turbulent} = 0.023 * Re^{0.8} (Pr)^{0.4}$

► **Friction factor correlations (no central channel)**

- Katheder correlation: $f = \alpha + \beta * Re^{-\gamma}$

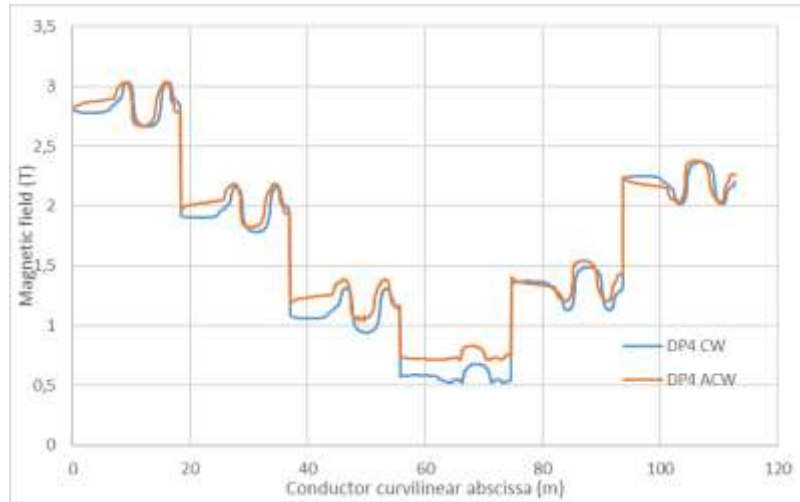
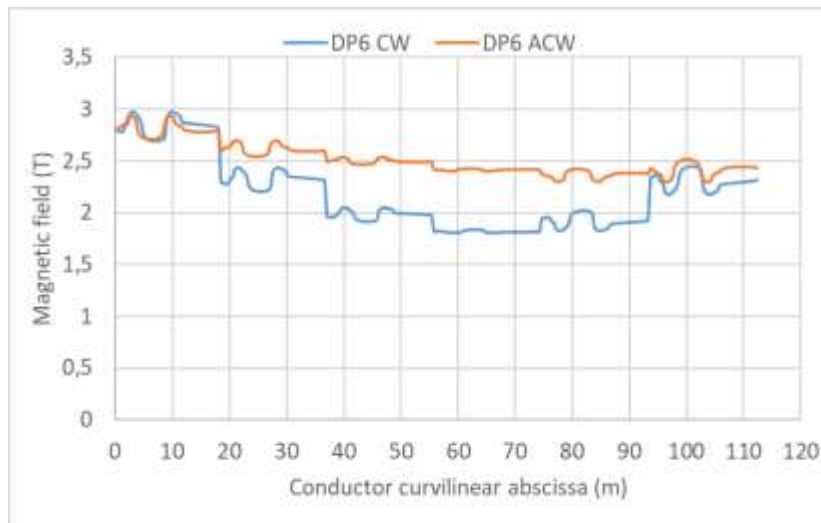
« α, β, γ **empirically fitted** at the OTHELLO test facility (CEA Cadarache, France) for all the CICC's that are used for the TF coils manufacture »

Cross-section of the CICC for JT-60SA TF Coils

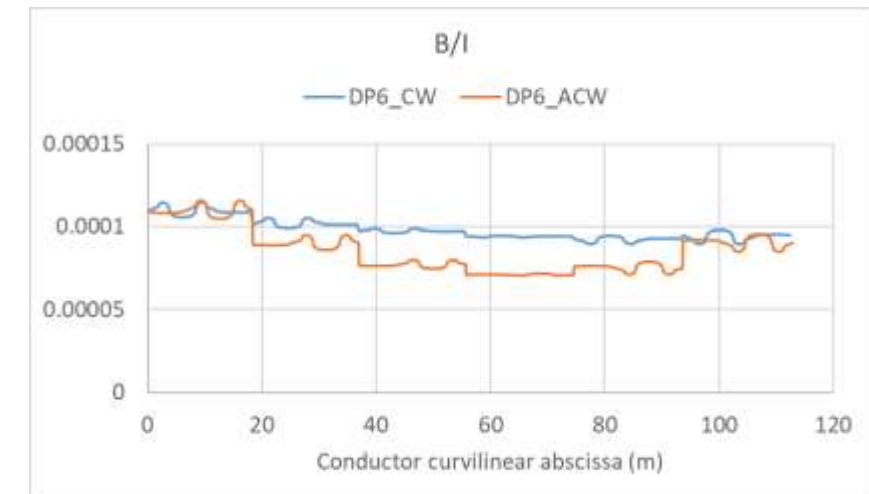
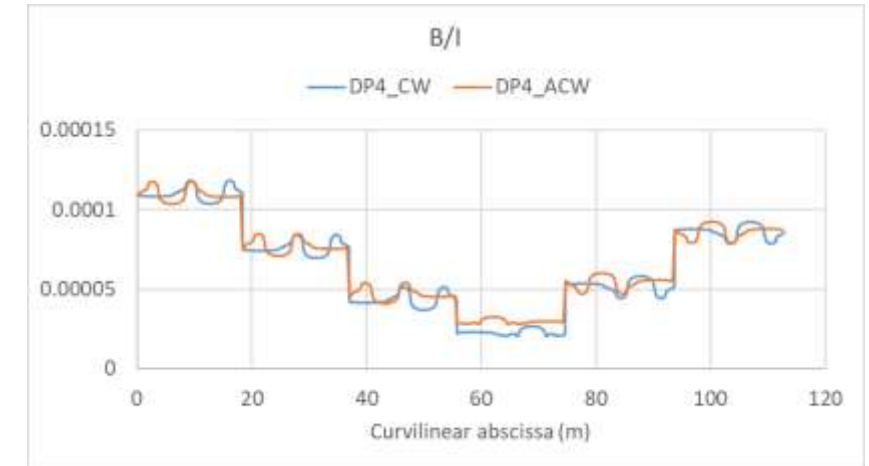


Schematic diagram of the forced helium convection in the unfolded CICC

Nominal current (25.7 kA)

CENTRAL pancake
DP4SIDE pancake
DP6

B/I ratio



3.a. Thermohydraulic code THEA: steady cryogenic simulation for TFC02

Friction factor and deposited heat before coil energizing

1. Average Heat load due to friction computed in THEA :

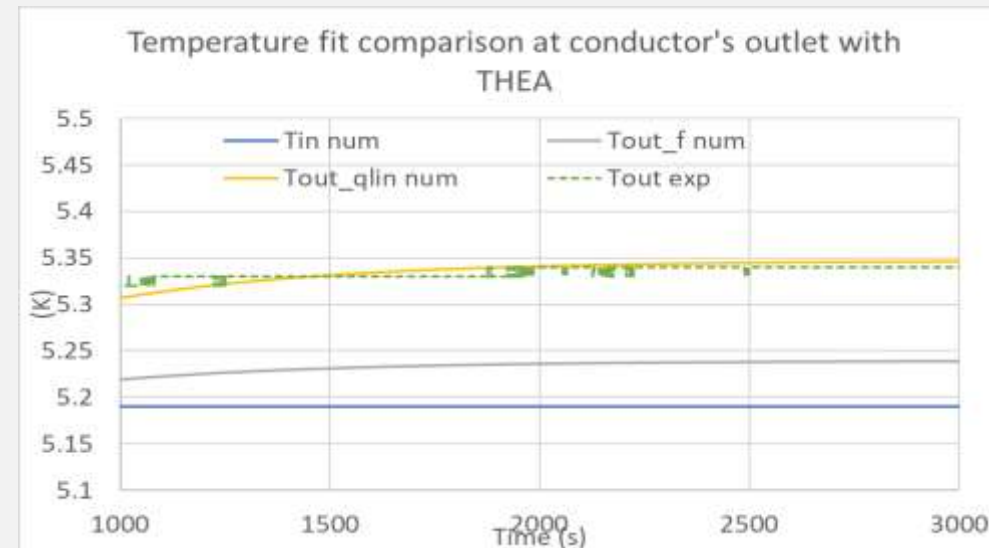
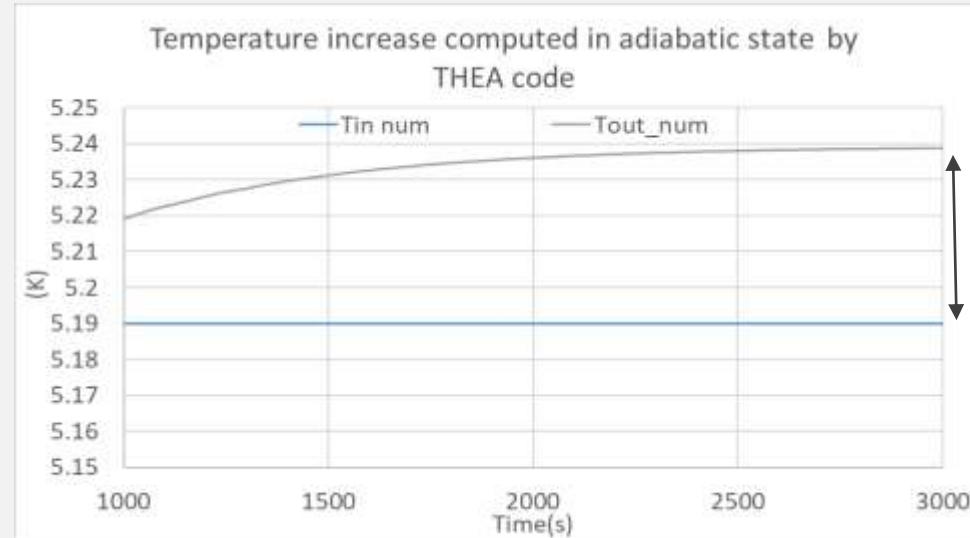
$$q_f = 3.53 \cdot 10^{-3} \text{ W/m}$$

$$(f(3853) = 0.1817)$$

2. Average heat load deposited in the Casing:

$$q_{lin} = 0.01 \text{ W/m}$$

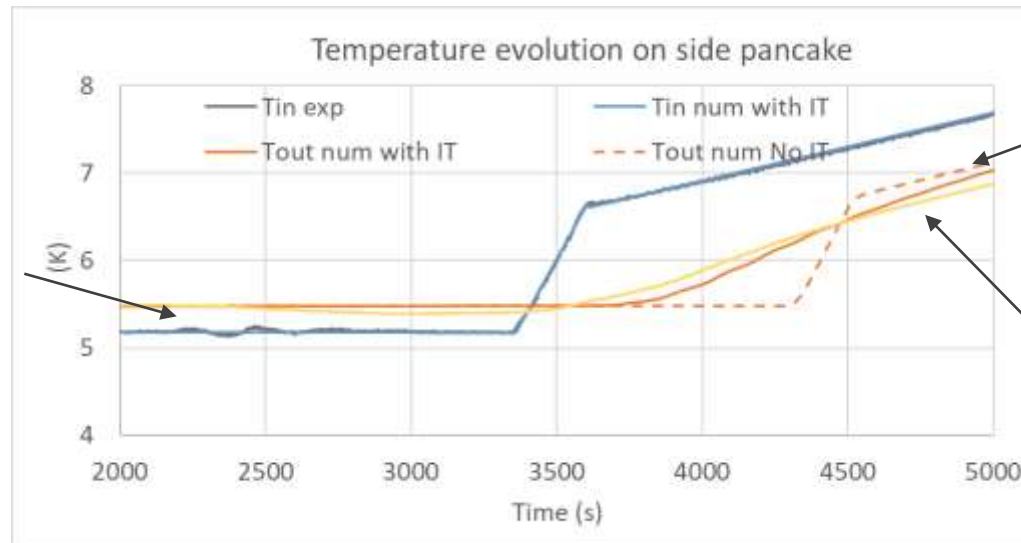
Computed from $(Q_{tot})^{exp} - q_f^{num}$



Computation of conductor's outlet temperature

► ATA10: lower current quench test

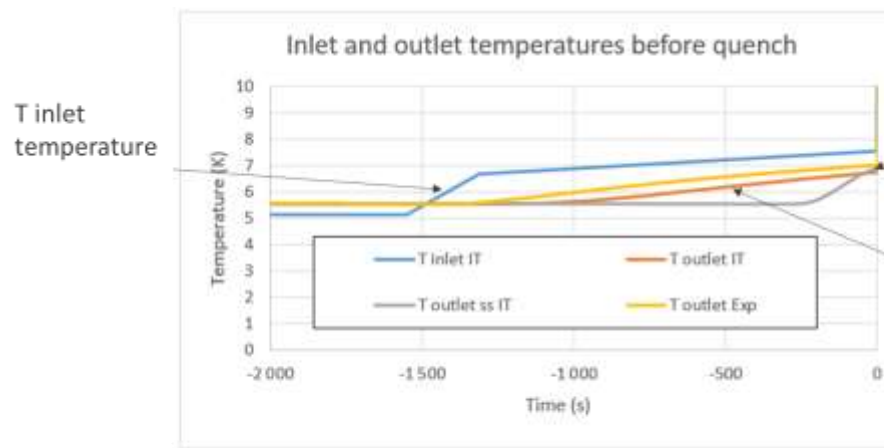
Experimental T_{inlet} temperature in WP and THEA computed inlet temperature



Outlet temperature using THEA model. NO interturn

Outlet temperature using THEA model with interturn and experimental outlet temperature

► ATA06: lower mass flowrate quench test



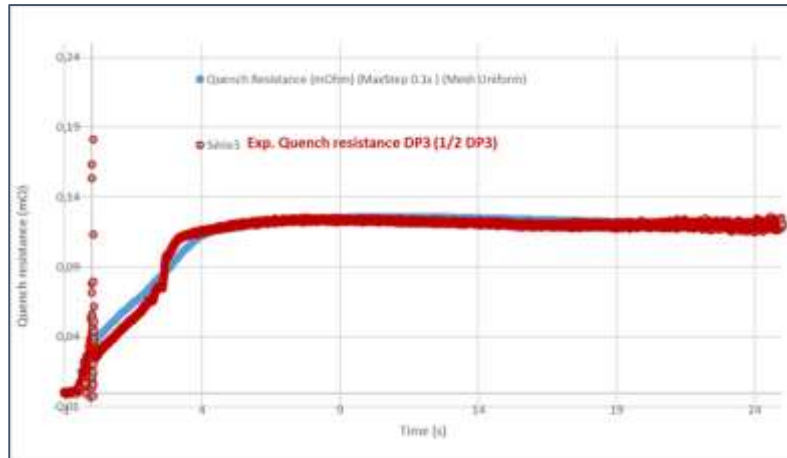
Thea calculated outlet temperature Without InterTurn Time decay due to mass flow time decay

T outlet experimental temperature and Thea calculated outlet temperature with InterTurn

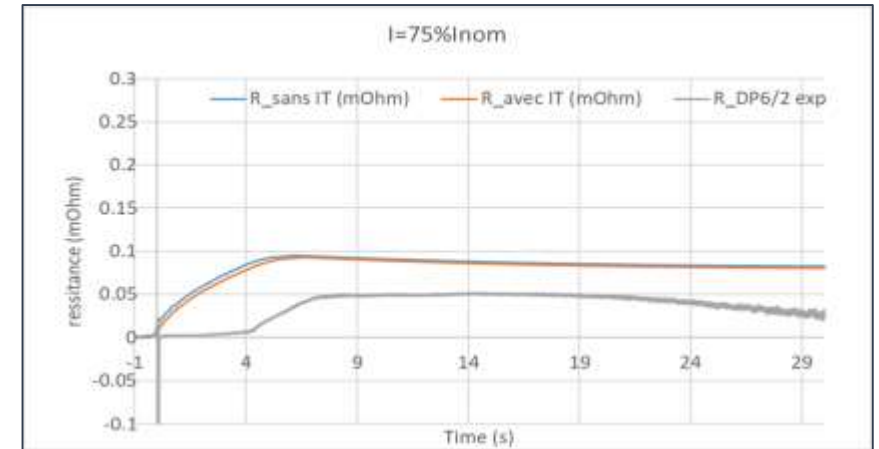
3.c. Numerical analysis of the experimental quench dynamics

Quench resistance evolution computed with THEA code of a single conductor (1 pancake); variant physical parameters according to 3 different quench test case at CTF.

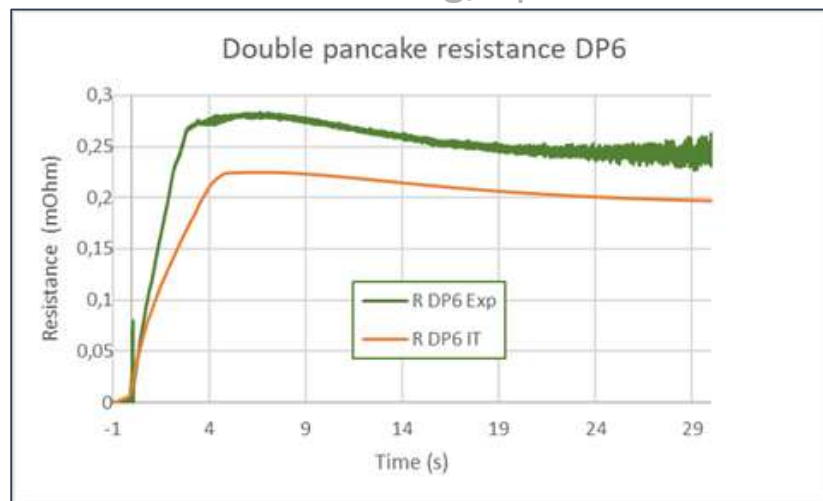
✓ Detection Time= 500 ms



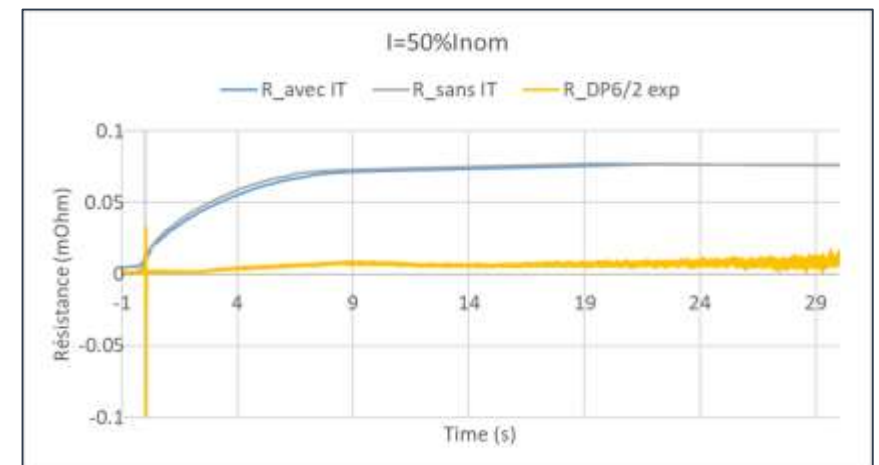
✓ Electrical Current = 75% I_{nom} (19.5 kA)



✓ WP flowrate = 1.5 g/s per conductor

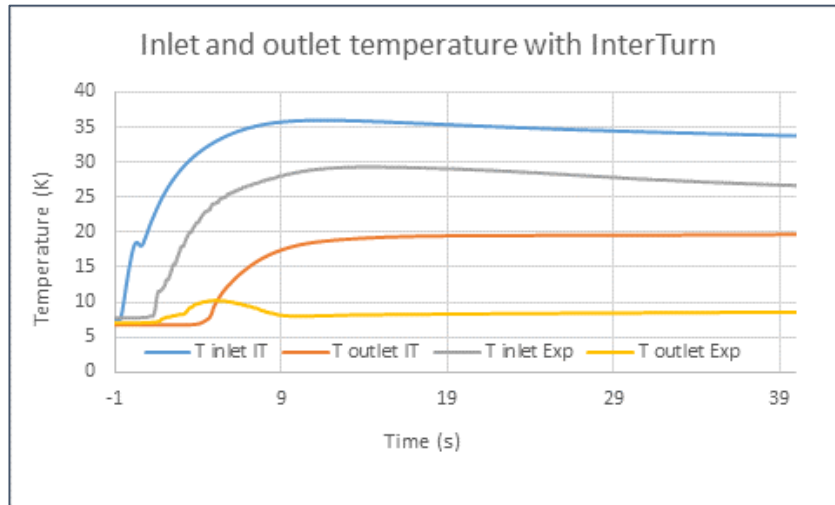


✓ Electrical Current = 50% I_{nom} (12.3 kA)

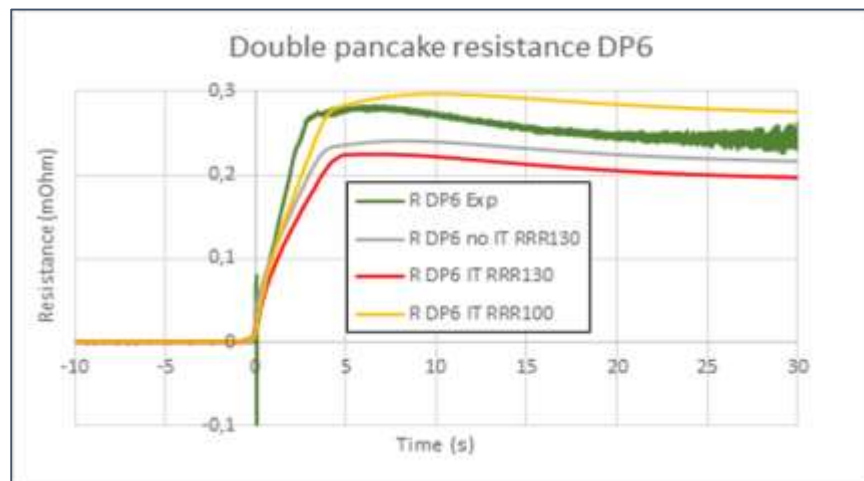


Present simulation limitations

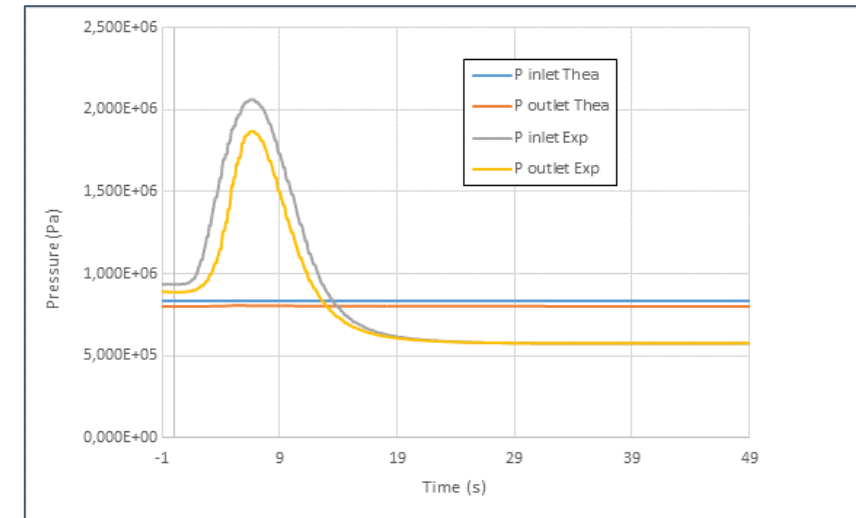
- Temperature



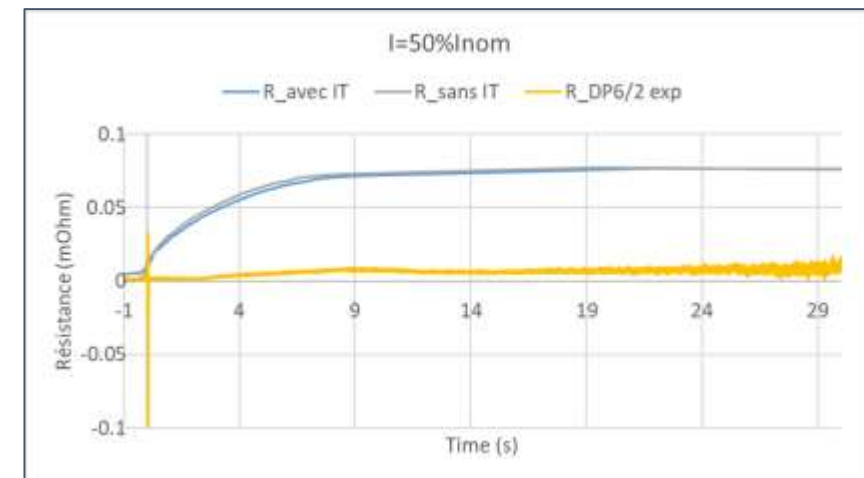
- Copper characteristics?



- Pressure? BC = infinite volume



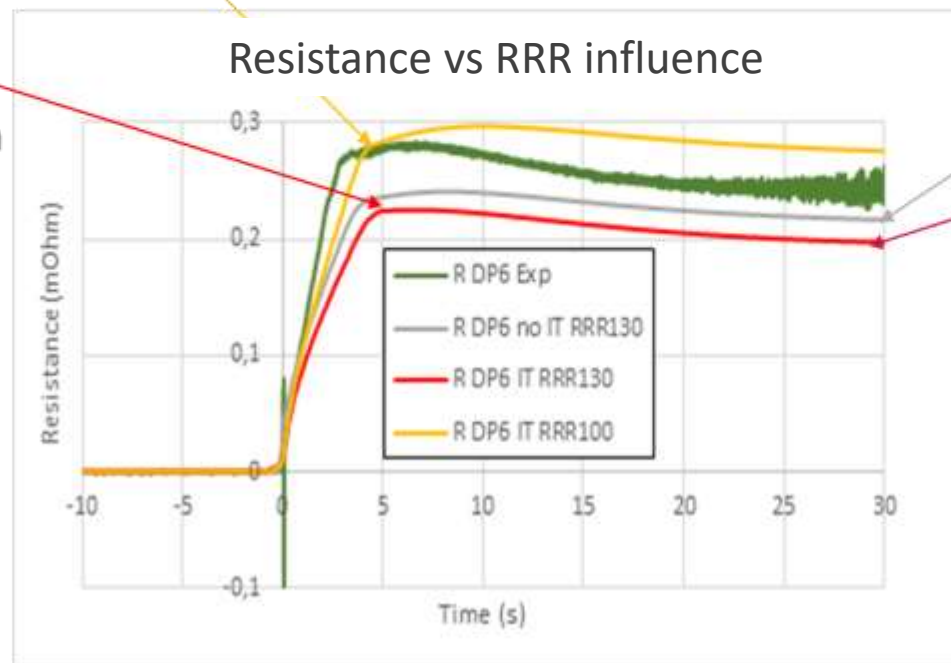
- Interpancake vs interturn influence?



Lower cooling massflow rate (1.5 g/s per conductor): interturn and RRR influence

Resistance vs RRR
influence

The difference
between RRR130
and RRR100 can
be estimate to
32% at maximum
point



Resistance vs Inter-turn
influence

The influence of
InterTurn can be
estimate to -7%
at maximum
point

- ▶ An additional experimental activity was conducted in order to perform further measurements on the two TFC spares.
- ▶ A database of 6 quench tests measurements is available for developing numerical models for the CICC.
- ▶ Experimental analysis and modelling are conducted for 3 different quench scenarios:
 - $\tau = 500ms$ for quench detection (longer than the nominal $\tau = 100 ms$)
 - Lower cooling massflow rate = 1.5 g/s instead of 2 g/s
 - Lower quench current (19.5 kA and 12.3 kA)

THEA simplified 1D code accounting for transverse heat transfer, friction factor influence and deposited heat provides good accuracy in **steady** state case and in **slow** transient state.

- ▶ Interturn model appears occasionally less influential in case of lower quench current, whilst conductor's material properties seems more important in case of a less efficient cooling of WP.

► Questions?

MERCI POUR VOTRE ATTENTION

- ▶ [1] Y. Huang, « Study and modelling of the thermohydraulic phenomena taking place during the quench of a superconducting magnet cooled with supercritical helium. », Plasma Physics [physics.plasm-ph]. Université Paris-Saclay, Saclay, 2018.
- ▶ [2] Huang Y. et al, "Quench tests analysis of the first JT-60SA Toroidal Field coils", 2017 *Fusion Eng. Des.* 124 147-152
- ▶ [3] L. Bottura et al. , A numerical model for simulation of quench in the ITER magnets, *Journal of Computational Physics* 125, 26-41, N°0077
- ▶ [4] P. Decool, H. Cloez, G. Jiolat, M. Tena, L. Zani, C. Hoa, W. Abdel Maksoud, and M. Verrechia. "JT-60SA TF Coils: Experimental Check of Hydraulic Operating Conditions". *IEEE Transactions on Applied Superconductivity* 26, June. 2016. doi: {10.1109/TASC.2016.2520585} (see pp. 3, 16, 17, 32, 102)